## Outline

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## Introduction

- Lesson learned from Fukushima accident, we need an alternative core cooling method to against severe nature hazard
- Solution to setup portable equipment, or called FLEX
  - Cheap, Easy to setup
- The effectiveness of FLEX strongly depends on the characteristic of the accident scenario
  - Procedure to lineup portable equipment
  - Hours of temporary core cooling
  - Failure probability of FLEX
- This paper provide an easy and conservative way to show the risk effectiveness of FLEX
- Examples for risk reduction evaluation
  - Estimate CDF reduction for different plant FLEX strategy

# FLEX strategy in Taiwan

- Develop "Ultimate Response Guideline"(URG) to keep fuel from damage during severe accident
- Entry point of URG (any of the follow conditions)
  - Loss of all core cooling ability (except steam driven cooling method)
  - Loss of all AC power including all standby diesel generators and gas turbine
  - Strong earthquake with tsunami alarm
- 3 stages of URG

	Stage	Purpose	Action Time
	1	Controlled depressurization and provide temporary core cooling (steam driven cooling or fire water)	< 1 hour
	2	Provide alternative core cooling	< 8 hours
-	3	Restore service water system , also long term plant cooling	< 36 hours

# **FLEX** strategy in Taiwan

#### • URG is designed to be an event base FLEX Strategy

- Controlled core depressurization to keep turbine driven pump alive
- Temporary core cooling to lineup portable long term power supply and cooling water supply

#### Success of URG (Plant damage Status)

- Main control room available for controlled depressurization
- Temporary core cooling (steam driven cooling or fire water)
- Good procedure to setup FLEX equipment in time
- Time to lineup of FLEX equipment (Building damage, road damage)



#### **Risk reduction estimation**

#### • Risk Reduction = $\sum_i (1 - P_i) \times CDF_i$

- $CDF_i$ : core damage frequency of  $i^{th}$  accident sequence
- $P_i$ : FLEX failure probability for  $i^{th}$  accident sequence
- Steps for risk reduction estimation
  - 1. Define risk significant initiating events and sequences
  - 2. Define plant damage status for each sequence
  - 3. Define proper FLEX probability of each sequence
  - 4. Calculate risk reduction for each sequence
  - 5. Summation of risk reduction

# Plant damage status categorization Main Control Core Damage Sequence Room 核能研究所 Category I~IV : Failure probability of FLEX will be given Category V : No credit for FLEX strategy

# **Failure Probability of FLEX**

Failure Probability	Remarks
1	No credit for FLEX Strategy
0.4	Generic failure probability form USNRC SECY-15-0085
0.1	Dominate by human error with less time available to lineup FLEX equipment
0.01	Dominate by human error with longer time available to lineup FLEX equipment
0.001	Significant long time available to lineup FLEX equipment. Human error is negligible. Component failure was used.

# **Risk significant IE**

Reactor	Inte	rnal Event	ts	External Events			
Туре	Transients <sup>1</sup>	Flood	Fire	Strong Wind	Seismic	Tsunami	
BWR-4	<u>18.0%</u>	2.2%	<u>28.4%</u>	<0.1%	<u>50.1%</u>	1.3%	
BWR-6	<u>19.2%</u>	8.9%	7.5%	<0.1%	<u>64.3%</u>	0.2%	
PWR	<u>13.9%</u>	1.1%	4.2%	2.7%	<u>69.7%</u>	<u>8.4%</u>	

\* This table reflects the risk with no FLEX available;

\* BWR: Boiling Water Reactor: PWR: Pressurized Water Reactor

<sup>1</sup> Transients include all kind of loss of coolant accidents and various system failures

# **CASE Study**

Case	Ac	cident S	equence	e Catego	ory	Note	
Case	I	II	III	IV	V	NOLE	
Case 1	0.4	0.4	0.4	0.4	1	Generic FLEX failure probability was used	
Case 2	0.01	0.1	0.1	0.4	1	Plant with pool or unverified FLEX procedure	
Case 3	0.001	0.01	0.1	0.4	1	Plant with high quality FLEX procedure	



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#### **Results of risk reduction**

#### • Symptom based FLEX procedure was assumed

#### By total CDF

Reactor Type	Case 1	Case 2	Case 3
BWR-4	35.6%	50.3%	50.6%
BWR-6	29.7%	44.2%	44.3%
PWR	24.5%	35.2%	35.4%

#### By CDF of each IE

Casa	BWR-4			BWR-6		PWR		
Case	Transients	Fire	Seismic	Transients	Seismic	Transients	Seismic	Tsunami
Case 1	32.6%	12.9%	54.5%	26.8%	73.2%	16.2%	69.9%	13.9%
Case 2	32.1%	11.4%	56.5%	27.8%	72.2%	13.2%	72.3%	14.5%
Case 3	32.1%	11.8%	56.2%	27.9%	72.1%	13.1%	72.3%	14.5%

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# **Risk profile wo/w FLEX**

• FLEX strategy may change the risk insight of PRA

#### Without FLEX Strategy

Reactor	Inte	ernal Even	ts	External Events			
Туре	Transients	Flood	Fire	Strong Wind	Seismic	Tsunami	
BWR-4	<u>18.0%</u>	2.2%	<u>28.4%</u>	<0.1%	<u>50.1%</u>	1.3%	
BWR-6	<u>19.2%</u>	8.9%	7.5%	<0.1%	<u>64.3%</u>	0.2%	
PWR	<u>13.9%</u>	1.1%	4.2%	2.7%	<u>69.7%</u>	<u>8.4%</u>	

#### With high quality FLEX strategy

	Reactor Type	Internal Events			External Events			
		Transients	Flood	Fire	Strong Wind	Seismic	Tsunami	
n	BWR-4	3.7%	4.4%	45.3%	<0.1%	43.9%	2.7%	
	BWR-6	12.2%	16.0%	13.4%	<0.1%	58.1%	0.3%	
-	PWR	14.3%	1.7%	6.5%	4.2%	68.3%	5.1%	

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#### **Results of other case study**

#### • FLEX strategy is risk effective in all cases

		gy was develop tsunami events	Plant with negligible external event risk				
Reactor	Event B	ased FLEX Pr	ocedure	Negligible External Event Risk			
Туре	Case 1	Case 2	Case 3	Case 1	Case 2	Case 3	
BWR-4	19.4%	28.4%	28.4%	34.9%	47.1%	47.7%	
BWR-6	21.6%	31.9%	31.9%	41.4%	64.0%	64.5%	
PWR	20.5%	30.5%	30.8%	28.7%	33.4%	33.5%	
	· <b>"</b> !						

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## Conclusion

- A simplified methodology was developed in this study to account for the risk effectiveness of different kind of FLEX strategy
- Predict the conservative risk reduction by examining the accident sequences from the event trees of plant specific PRA
- The results suggested that FLEX can significantly decrease CDF no matter it is event based or symptom based
- Even for those plants with negligible external event risk, FLEX is still an effective way to significantly reduce plant risk
- It is important that a specific FLEX procedure should be developed to provide guidance for both reactor operator and other operating crew
- Implementing FLEX may change the risk insight of plant PRA

# **Question & Comment**