



PSAM 14

Probabilistic Safety Assessment and Management

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Uncertainty Analysis for Input Parameters of Electrical Cabinet Fire Simulation by Coupling Latin Hypercube Sampling and CFAST

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- 1 Introduction
- 2 Fire Modeling
- 3 Results
- 4 Conclusion

1 Introduction



- **Fire risks in nuclear power plants**
The safety of reactor core
- **Main control room**
control center; operators' lives and facilities' functions
- **Fire modeling**
fire risk evaluation tool
- **Parameter uncertainty for fire modeling**
uncertainty analysis

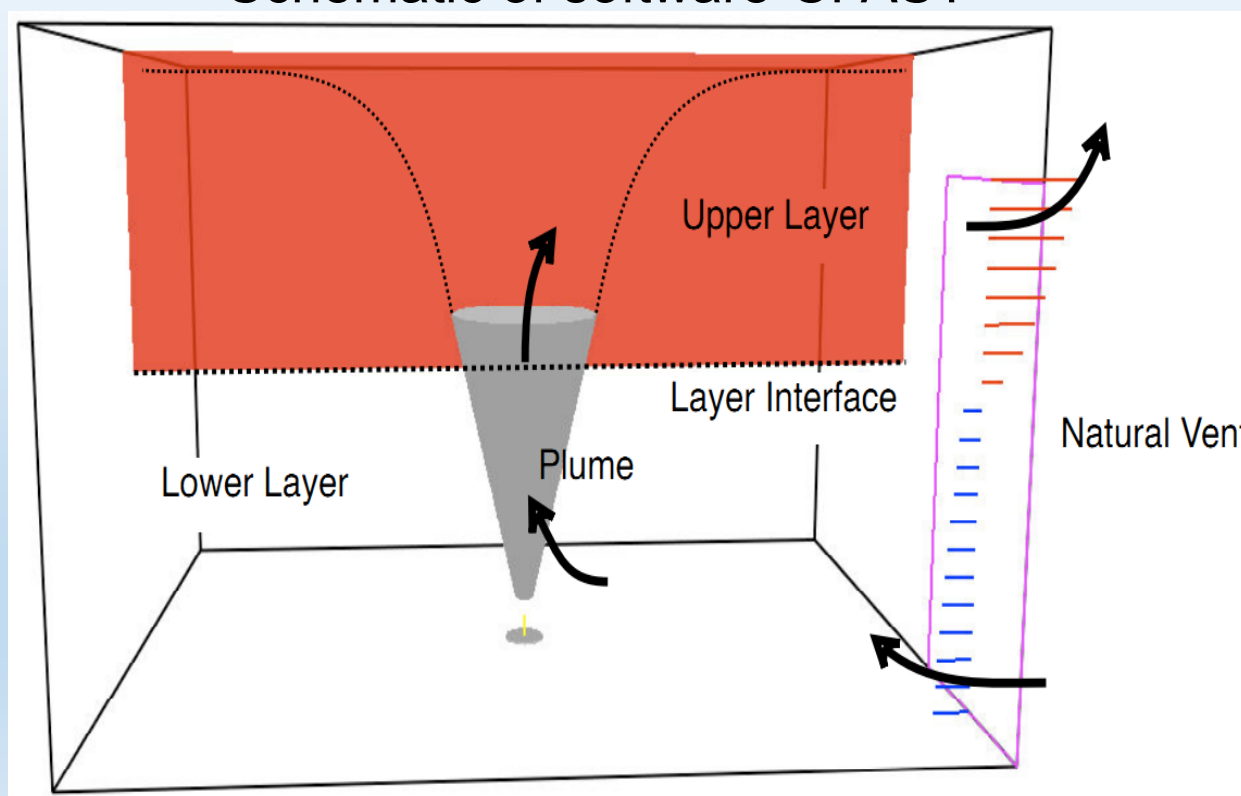


Objective:

Based on electrical cabinet fire scenarios in a main control room (MCR), parameter uncertainties were evaluated by coupling Latin Hypercube Sampling and CFAST



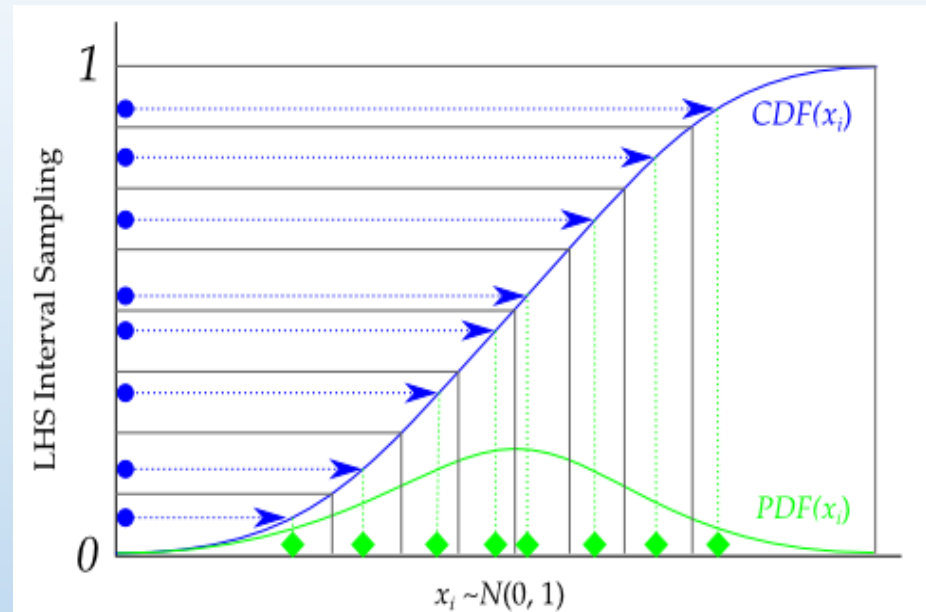
Schematic of software-CFAST





Advantage:

Drastically reduce the number of runs necessary to achieve a reasonably accurate result



Procedure:

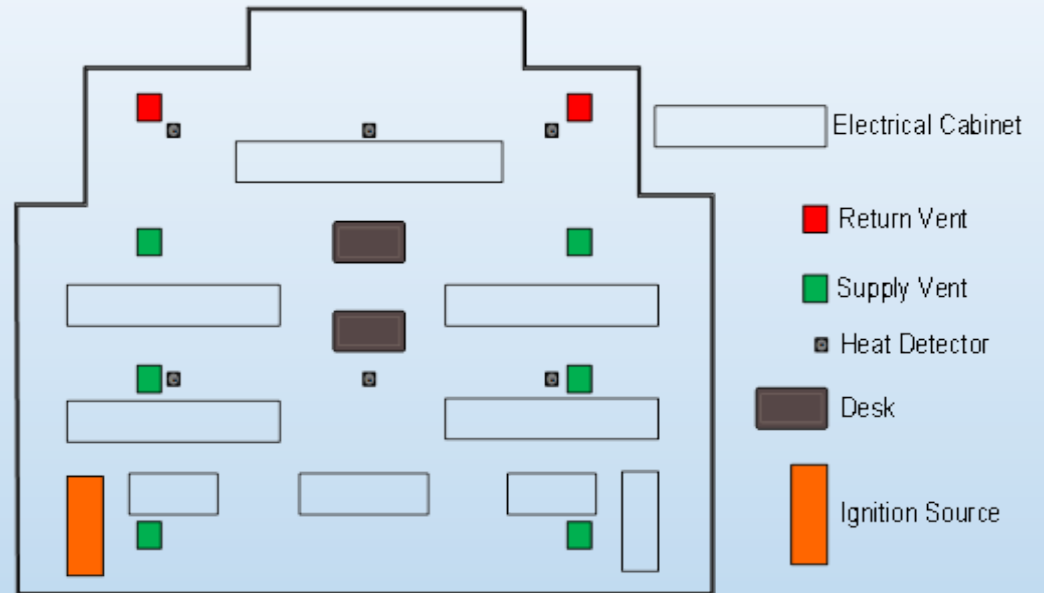
- ① split the $[0, 1]$ interval into N equiprobable intervals
- ② propagate via the inverse CDF to the output distribution
- ③ take N standard samples from each interval of the output distribution

2 Fire Modeling



Layout:

- ① irregular structure
- ② electrical cabinets
- ③ 6 supply vents and 2 return vents
- ④ 6 heat detectors installed



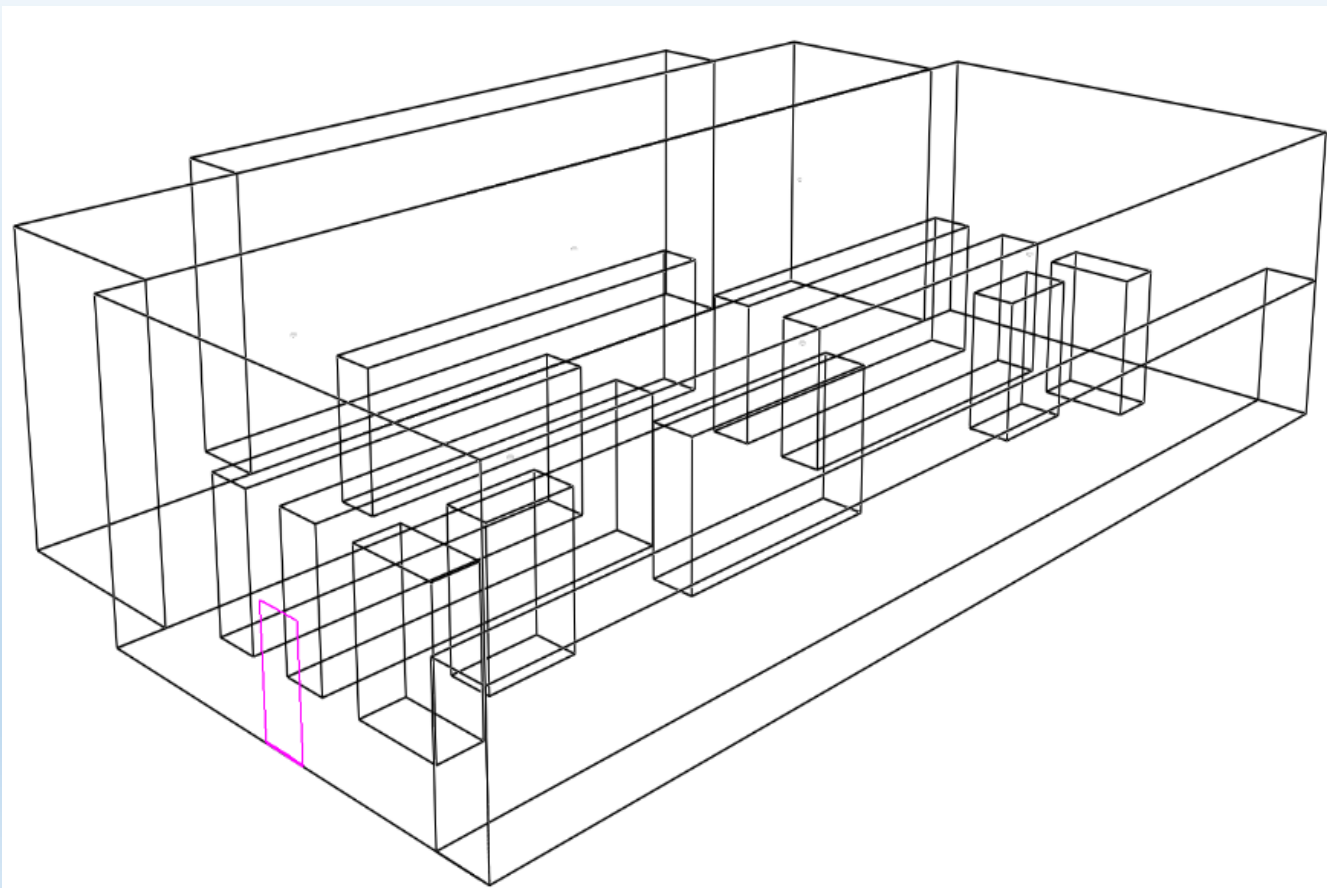
Assumptions:

- ① door is assumed to be closed
- ② operators is assumed to be between two desks
- ③ fire source is modeled 0.3 m below top surface of the ignited electrical cabinet



Fire Modeling

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➤ **Heat release rate**

Energy released per second

➤ **Soot yield**

mass of soot produced per unit mass of fuel consumed

➤ **Activation temperature**

temperature at which detectors activate

➤ **Response time index**

feature how fast detectors response in fire scenarios



Parameter	Distribution	Parameter1	Parameter2
Heat Release Rate ¹	Gamma	$\alpha=0.7$	$\beta=216$
Soot Yield ²	Gamma	$\alpha=2.375$	$\beta=0.047$
Activation Temperature ³	Normal	$\mu=80$	$\sigma=10$
Response Time Index ³	Normal	$\mu=0.25$	$\sigma=0.03$

[1] EPRI, NRC-RES. NUREG/CR-6850 EPRI/NRC-RES Fire PRA Methodology for Nuclear Power Facilities Volume 2: Detailed Methodology [R]. U.S. Nuclear Regulatory Commission and Electric Power Research Institute, 2005.

[2] Tom Elicson, Jim Bouchard, Heather Lucek, Bentley Harwood. Calculation of Fire Severity Factors and Fire Non-Suppression Probabilities for a DOE Facility Fire PRA [C]. ANS PSA 2011 International Topical Meeting on Probabilistic Safety Assessment and Analysis. Wilmington: ANS, 2011

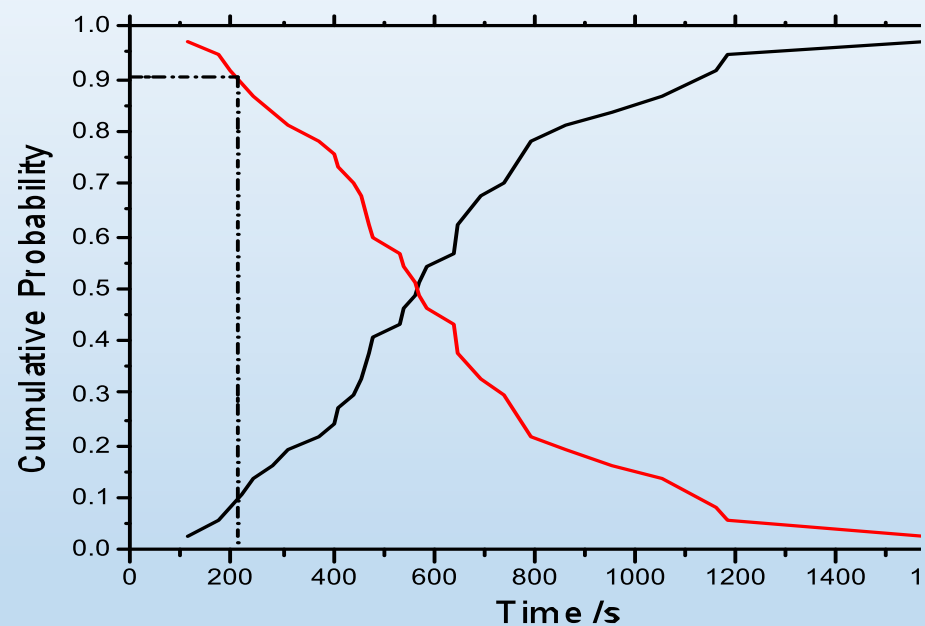
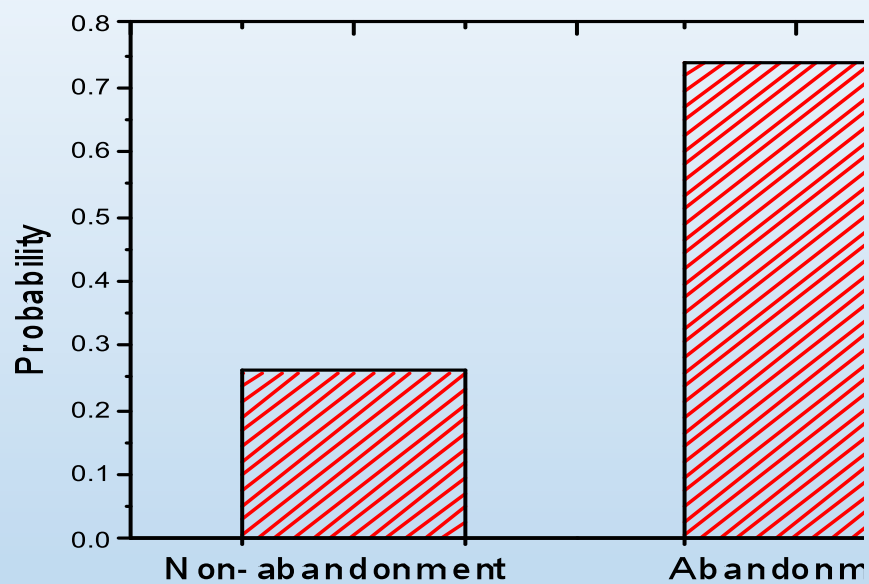
[3] Jia Jia. Study on Assessment Methodology of Ship Fire survivability [D]. Hefei: University of Science and Technology of China, 2014.

3 Results



Results

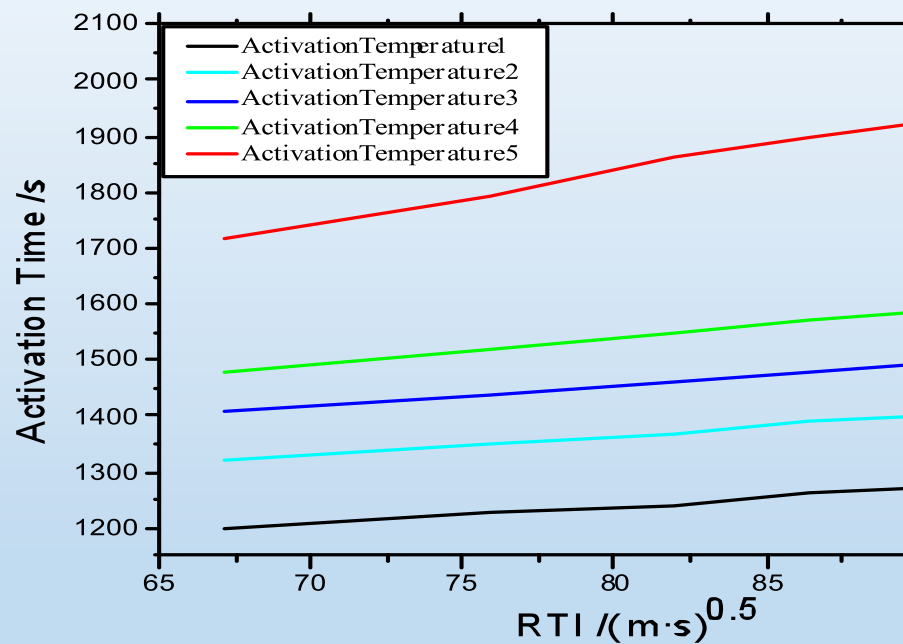
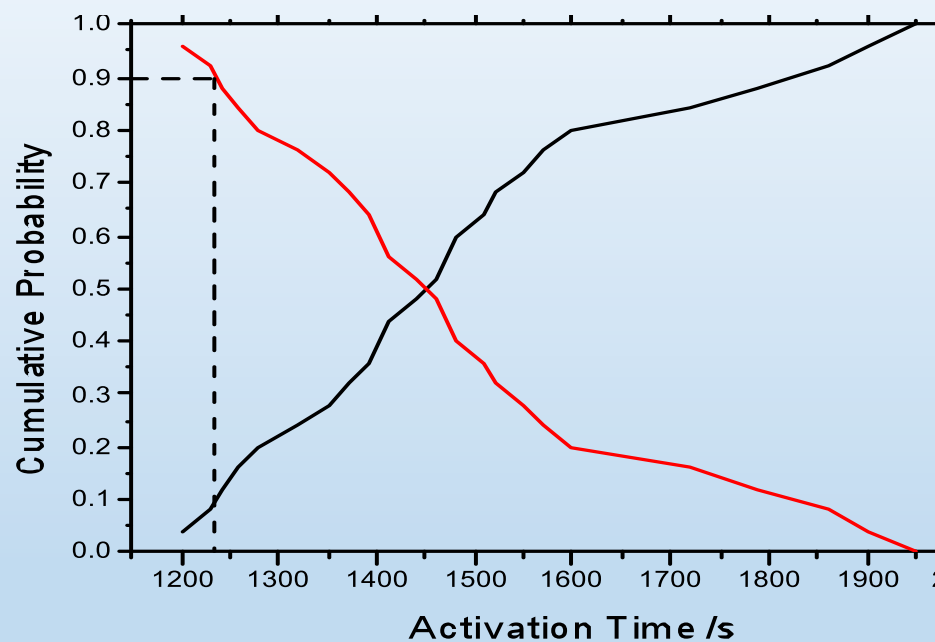
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Results

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non-suppression probability is an estimate of the overall likelihood that given a fire scenario in the postulated fire ignition source, the damage to the target set will occur before the fire is finally suppressed. It can be calculated by the following equation:

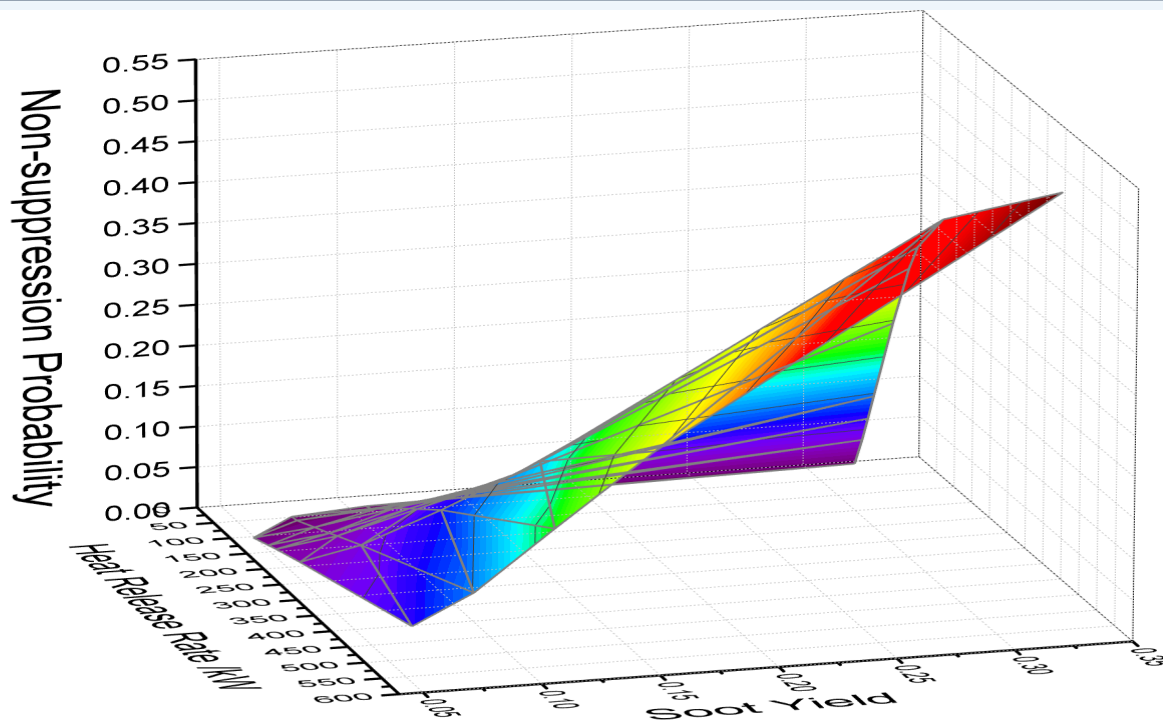
$$NS(t) = e^{-\lambda t}$$

where λ is the suppression rate constant, 0.33/min for the MCR fire. t is the time from fire ignition to operators abandoning the MCR.



Results

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4 Conclusion



- Heat release rate and soot yield, the most important factors for fire simulations, can lead to the MCR's inhabitability.
- Statistical results show that abandonment probability is 74% and the 0.9-quantile for complementary cumulative probability curve is around 220s.
- Activation time is heavily affected by activation temperature and response time index.
- Non-suppression probability increases with heat release rate and soot yield increasing.

THANK YOU!

Presented by Changhong PENG